

Neutron Activation Analysis of Cement Bulk Samples

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ABSTRACT

Studies of Neutron Activation Analysis (NAA) have been carried out on different Egyptian cement samples. Two Am-Be isotopic neutron sources of activity 5 Ci were used in this investigation. The accomplished gamma rays were measured using 70 % HPGe spectrometer. This work demand to estimate the elements contained in cement products and its quality control. X-Ray Fluorescence (XRF) measurements were done for confirming our results, and for determination the average neutron flux of $3.7 \times 10^3 \text{ n/cm}^2 \text{ sec}$. The Natural radioactivities of these samples were measured before the analysis to know the background level of ^{40}K , ^{238}U and ^{232}Th nuclei. The results investigated that NAA agree with the results of XRF and the world range of the cement concentration of the essential elements Ca, Al, Na, Fe, Mn, V, Sr and Si.

Keywords: NAA / XRF/ Natural Radioactivity / Cement.

INTRODUCTION

A several studies [1, 2] were leading to the launching of neutron activation analysis. These studies were linked to a topic of the day which even at present is a challenging analytical task: the determination of a rare earth in one of the other elements of the rare earth series. The analyst would make use a transfer system for short and long irradiations in reactor sites with a suitable thermal neutron.

Neutron activation analysis is the most powerful analytical technique for the quantitative and the qualitative analysis. The elements to be determined in a sample are made radioactive by irradiating the sample with neutrons. The number of detected γ -rays of a particular energy is directly proportional to the disintegration rate of the radionuclide, which in turn is directly proportional to the amount of parent isotope in the sample and can be determined by using the absolute method [3]

Determination of trace concentration of certain elements in cement is very useful in evaluating their suitability as building materials. The need for cement is so great because cement is the main

component that enters into the building construction. The drinking water are collected in concrete reservoirs then resolved to distill it from minerals. Depending on the chemistry of certain toxic or radioactive elements, some could leach into drinking water and could pose a health risk to the dwellers [4].

The most important naturally occurring radionuclide ^{40}K , ^{232}Th and ^{238}U in Cement as part of building material were investigated [5, 6]. These radionuclides are sources of external and internal radiation exposure in the buildings. Therefore, the dweller of houses and office buildings, where they spend about 80% of their time [7, 8], are exposed to radiation emitted by these radionuclides.

Gamma radiation originating from ^{40}K , ^{232}Th and ^{238}U series isotopes cause external exposure also the inhalation of the dangerous gases ^{220}Rn and ^{222}Rn [9, 10]. From the risk point of view, it is necessary to know the dose limits of known exposures. Simultaneously determination the natural environmental radiation level provided by these building materials, In order to assess the radiological hazards to human health, so it is important to study the radioactivity levels emitted by this material.

The aim objectivities of this work are to study NAA with thermal neutrons from isotopic Am-Be neutron source to qualify and quantify the presence of the elements in bulk cement samples from the local Egyptian market as well as to quantify the natural radionuclides in these samples.

MATERIALS AND METHODS

Cement samples were collected from which have been used in Egypt. The samples are 1.5 Kg for each weighted then dried and put into clean containers. Samples were stored for 30 days period to achieve the secular equilibrium then measure these samples 18000 sec. Indium foils were also included in the irradiation containers for monitoring the neutron flux variation along the vertical axis.

The gamma-ray system is consist of Canberra coaxial High- Purity Germanium detector (HPGe) which has a photo peak efficiency of 70% and energy resolution of 2keV full-width at half maximum (FWHM) for the 1332 keV gamma-ray line of ^{60}Co . A cylindrical lead shield of five cm thickness, which contains inner concentric cylinder of Cu with thickness of 10 mm, was used to shield the detector and to reduce the effect of background. The detector was cooled to liquid nitrogen temperatures and coupled to a PC-based 8K multichannel analyzer and an ADC with Genie 2000 for data acquisition and analysis.

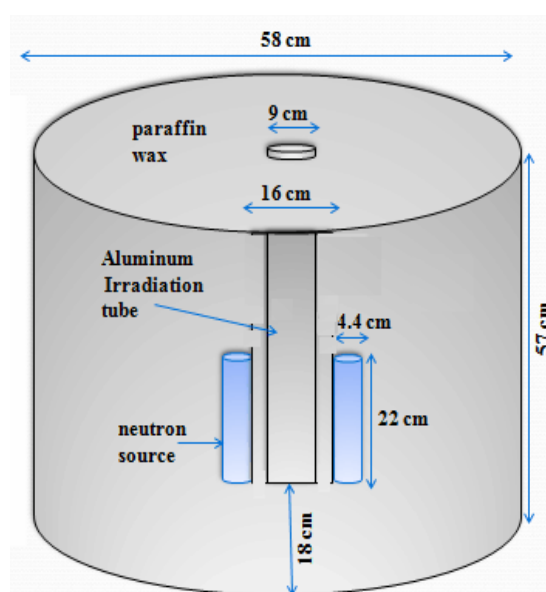
The calibration of the detector was carried out by using standard point sources (^{60}Co , ^{137}Cs , ^{22}Na and ^{133}Ba). Absolute efficiency calibration curves are calculated for activity determination of the sample by using standard ^{226}Ra , contained in the same cylindrical bottles as the samples. The standards and the samples were prepared with a uniform geometry. An empty bottle with the same geometry was measured for subtracting the background.

The gamma-ray transitions of energies 1120.3 keV (^{214}Bi) and 1764 keV (^{214}Bi) were used to determine the concentration of the ^{238}U series. The gamma-ray transitions of energies 911.1 keV (^{228}Ac) and 2614 keV (^{208}Tl) were used to determine the concentration of the (^{232}Th) series. The 1460 keV gamma-ray transition of ^{40}K was used to determine the concentration of ^{40}K in different samples as shown in table (1) and their intensities.

Table (1) the natural radionuclides, their gamma lines used and their intensities [11].

Parent	Nuclide	Daughter Nuclide	γ -ray energy (keV)	Abundance (%)
²²⁶ Ra		²¹⁴ Bi	1120.28	14.9
		²¹⁴ Bi	1764.52	16.07
²³² Th		²²⁸ Ac	911.16	29.0
		²⁰⁸ Tl	2614.7	36.0
⁴⁰ K			1460	10.67

The present facility plans to use two identical isotopic radioactive neutron sources of Am-Be type. Each source has a cylindrical shape with dimensions of about 22 cm long and 4.4 cm in diameter, and has an activity of about 5 curies (185 GBq). The sources are put in an iron cylindrical barrel which filled with melted pure paraffin where the sources are isolated from the inner vertical central irradiation tube with the same paraffin material as shown in figure (1).

**Figure (1): Describes the neutron irradiation facility used through the investigation.**

The cement analysis was performed by S2 Ranger x-ray spectrometer which consist of x-ray tube of pd mode and XFlash silicon drift detector of resolution < 145 eV at Mn K α . The samples are prepared in the pressed pellets with diameter of 20mm.

RESULTS

Natural specific activity measurement

The activity levels for radionuclides in the measured samples are computed using the following equation [12] :-

$$A = \frac{C}{PW\epsilon} \text{ (Bq / kg)} \quad (1)$$

Where:

A the activity level of a certain radionuclide (Bq/ kg), C the net count rate of the sample (counts / seconds), ϵ the detector efficiency for the specific gamma ray energy, P the absolute transition probability of gamma decay, W the weight of dried sample (kg). The activity concentration of the samples is shown in table 2 where all samples are lower than world average except sample S4 and S6 for ²³⁸U larger than 50 Bq/Kg.

Table (2) shows the activity concentration of different cement samples

Material	Mark	Activity concentration(Bq/Kg)		
		²³⁸ U	²³² Th	⁴⁰ K
Cement	S 1	42.77	21.9	74.67
	S 2	39.12	16.14	74.46
	S 3	37.95	18.33	145.33
	S 4	50.33	18.75	128.63
	S 5	42.74	16.27	120.92
	S 6	61.07	27.95	222.38

The radium-equivalent activity

The distribution of ²²⁶Ra, ²³²Th and ⁴⁰K in Cement is not uniform. Uniformity with respect to exposure to radiation has been defined in terms of radium equivalent activity in Bq/kg unit, to compare the specific activity of materials containing different amounts of ²²⁶Ra, ²³²Th and ⁴⁰K. The Radium-equivalent activity (R_{aeq}) was calculated from the following equation [13] :-

$$R_{aeq}=A_{Ra}+1.43A_{Th}+0.077A_K \quad (2)$$

Where A_{Ra} , A_{Th} and A_K are the activity concentrations of the three radionuclide's ²²⁶Ra, ²³²Th and ⁴⁰K, respectively, expressed in Bq/kg.

Radiation hazard index

Radiation hazard index, representative level index,
Defines as [13] :-

$$I_{yr}=(A_{Ra}/150 + A_{Th}/100 + A_K/1500) \quad (3)$$

This index can be used to estimate the level of radiation hazard associated with the natural radio nuclides in specific building construction containing cement. The Radiation hazard index varies between 0.3 and 0.5 lower than 1 which is the maximum suggested value as shown in table 3.

Table (3) the radium-equivalent activity (R_{aeq}) and Radiation hazard index of the External annual dose (EAD).

Material	Mark	R_{aeq} (Bq/Kg)	EAD(mSv/year)
Cement	S 1	77.87	0.34
	S 2	66.48	0.29
	S 3	73.70	0.33
	S 4	85.36	0.38
	S5	73.85	0.33
	S 6	115.65	0.51

Neutron activation analysis

Neutron activation analysis (NAA) was launched in Ref. (1), and has since then undergone a dramatic evolution with respect to instrumentation and applications. From a few selected examples throughout these years, it is concluded that NAA exhibits number of outstanding features which are of lasting value up to the present day. This should also be seen in the light of recent developments which are paving the way to more effective, new and specific applications. The absolute method is the most reliable for determination the elements in the samples with respect to their half-life's as shown in table 4.

Table (4) Elements determined nuclides and gamma ray energies measured [11].

The elements	Sample 1		Sample 2		Sample 3	
	NAA	XRF	NAA	XRF	NAA	XRF
Ca	87.3	73.83	63.86	71.01	68.19	71.50
Si	--	14.07	--	17.14	--	17.07
Al	5.59	--	5.95	4.13	4.60	3.77
Mg	--	--	--	1.70	--	2.51
Mn	0.23	0.34	0.07	--	0.13	--
Na	0.69	--	0.49	2.82	0.34	1.56
Fe	--	8.37	--	3.73	--	3.59
V	0.01	0.02	--	--	--	--
Sr	0.53	0.24	--	--	--	--

The analysis of the samples are performed by NAA and XRF where the results show that the concentration of Ca element varies from 63.8 to 87.3 in addition to other elements are found in the sample like Si, Al, Mg, Mn, Na, Fe, V and Sr as shown in table 5.

Table (5) the comparison between the analyses of NAA with XRF:

Element	Nuclide Used	Gamma lines measured, keV	Half-life
Ca	⁴⁹ Ca	3084.5	8.72 m
Al	²⁸ Al	1779.0	2.24 m
Na	²⁴ Na	1368.6,2754	14.96 h
Mn	⁵⁶ Mn	846.8,1810.7,2113.1	2.58 h
Sr	⁸⁷ Sr	388.4	2.81 h
V	⁵² V	1434.1	3.75 m

Present and future role of neutron:

The above given measurements represent only some applications reported since the launching of NAA. More information can for instance be found in the series of several proceeding and works in neutron activation analysis e.g. Ref. [1, 2, 9 and 19], from all this, it is clear that the success of NAA was due to number of outstanding features. Indeed, its excellent sensitivity for many element, its theoretical foundation enabling to identify and predict all sources of systematic or radon variation. And its analysis of e.g. Cement has been carried out, all these combined features made NAA to very powerful and reliable analytical tool.

In spite of these top-level performances, it should not be denied that in recent years NAA was seriously waylaid by a number of developments and circumstances. Indeed one should use NAA for what it can do, and one should realize that some of its features are of lasting value and even unique up to the present day. The present and feature role of NAA should be interpreted in the light of new insights into its power. This especially refers to the fact that it is an accurate and traceable method, with a physically different basis compared to other techniques, and with a built in possibility of self verifications since in many cases results for an element can be obtained via different gamma lines of different isotopes Ref [15, 14]. This makes NAA to a references method, which is eminently suited for the calibration of other analytical techniques Ref. [16] for the certification of references materials.

It is believed that the above measurements will lead to a situation where NAA will continue to fulfill its specific and valuable role in the arsenal of the available analytical techniques in table 6.

Table (6) The Comparison of elements concentration of the world cement with the local cement Measured by the available techniques:-

Technique	XRF	PGAA	NAA	XRF	NAA
Elements	[17]	[18]	[19]	This study	This study
Ca	67.5	64.0	63.2	73.8	87.3
Si	21.6	22.0	21.9	14.0	---
Al	4.5	4.0	5.1	---	5.59
Fe	3.6	2.2	2.8	8.37	---
Mg	1.5	0.21	2.3	---	---
Na	0.19	0.31	---	---	0.69
K	0.55	1.96	---	---	---
S	0.05	---	2.5	2.72	---
Ti	0.2	---	---	0.40	---
Mn	---	---	---	0.34	0.23
Sr	---	---	---	0.24	0.53
V	---	---	---	0.02	0.01

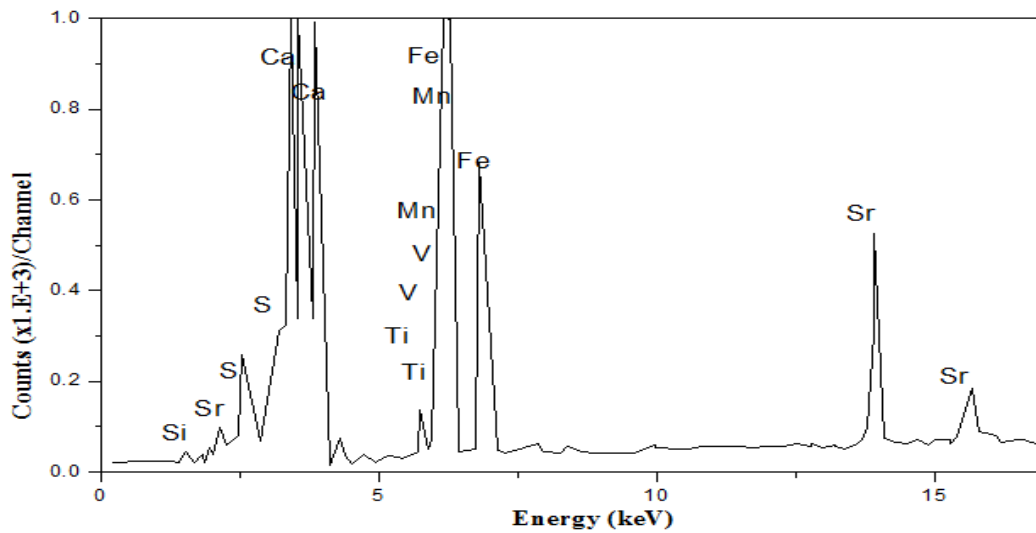


Figure (2): illustrates the analysis of cement sample by XRF Technique.

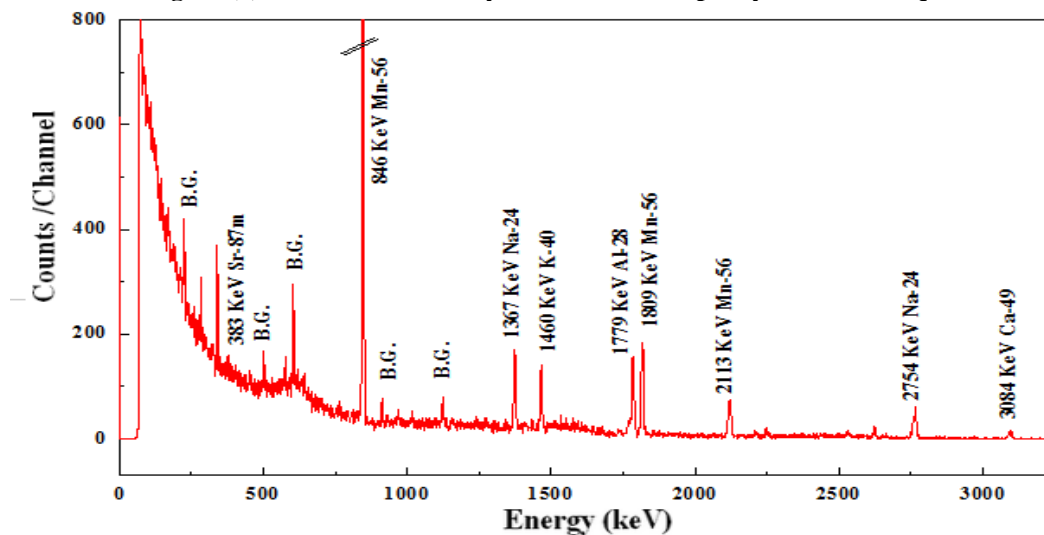
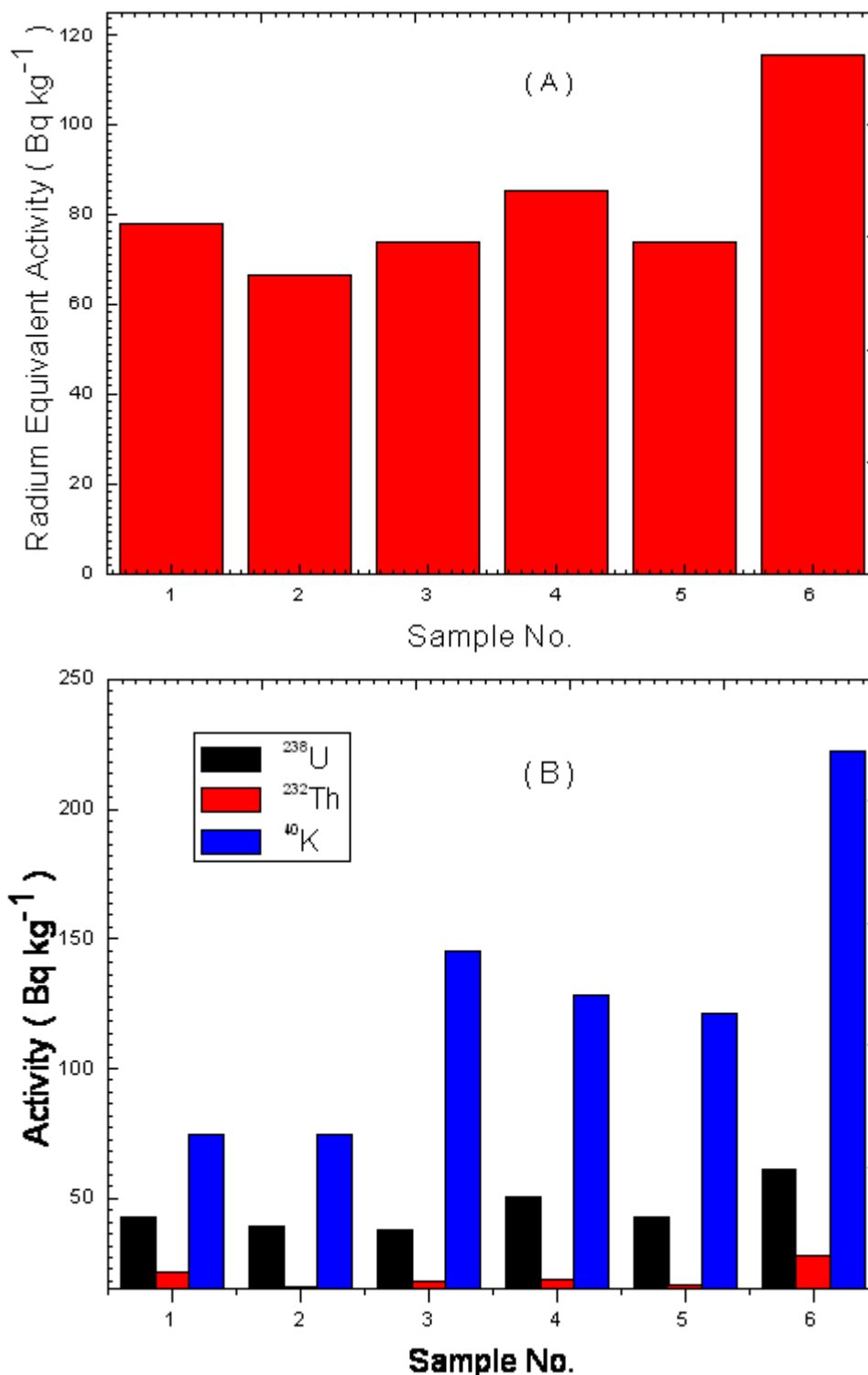


Figure (3): illustrates the analysis of cement sample by NAA Technique.



**Figure (4): (A) Radium equivalent activity in collected samples.
(B) The ^{238}U , ^{232}Th and ^{40}K concentration in different samples.**

The activity concentrations for ^{238}U , ^{232}Th , ^{40}K and Ra-equivalent activity are shown in Fig. (4A,B). The activity concentrations for ^{238}U ranged from 37.95 to 61.07 Bq Kg⁻¹, with an average of 45.65 Bq Kg⁻¹, ^{232}Th ranged from 16.14 to 27.95 BqKg⁻¹, with an average 19.89 BqKg⁻¹ and variation in ^{40}K activities ranges from 74.46 to 222.38 BqKg⁻¹ with an average 127.73 BqKg⁻¹, respectively.

CONCLUSION

In this work, a NAA method for the cement analysis has been carried out. An important parameter for the quality control of art building is the calcium proportion present in the cement. The neutron activation seems to be the more reliable and practical method to determine the concentration of the elements in bulk samples and nondestructively. Isotopic neutron sources like Am-Be, has sufficient flux for the qualitative and the quantitative neutron activation analysis of bulk samples with large amounts in the range of kilograms.

Further, this method is easily applied if minimal condition for radioprotection is respected and the cost is reasonable over destructive techniques is taken into account.

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